

INVESTIGATION OF CHANGES IN BACKGROUND RADIATION DUE TO TECHNOSPHERE

OBJECTS

Mathias Zulu¹, G.A. Yakovlev², V.S Yakovleva¹

¹National Research Tomsk Polytechnic University,

Russia, Tomsk, Lenin Avenue, 30, 634050

² National Research Tomsk State University,

Russia, Tomsk, , Lenin Avenue, 36, 634050

E-mail: chamzgalary@gmail.com

The gamma-background of the urban atmosphere is formed to a greater extent by the radiation of radionuclides contained in the soil, building materials, and the atmosphere. The influence of various objects of the Technosphere has practically not been studied by anyone. It is not known which objects will increase the total urban gamma background, and which ones will decrease. The foregoing determined the main goal of this work - the study of the influence of Technosphere objects on the gamma background radiation of the urban environment. The study was carried out in the city of Tomsk, Russia. Background radiation was studied using highly sensitive intelligent gamma detectors BDKG-03. It was determined that, within a radius of 1m from certain Technosphere objects the absorbed dose was 1.5 to 4.4 higher than the UNSCEAR recommended safe limit. The highest recorded dose for a person standing 50cm away from the technosphere objects was 204 nGy/h which is 2.4 times higher than the recommended safe limit. The range of absorbed dose was 84 nGy/h to 374 nGy/h. The highest calculated range of AEDE was 0.17 to 0.57 mSv/yr and ELCR was $0.59 \cdot 10^{-3}$ to $2.01 \cdot 10^{-3}$.

MCNP SIMULATIONS OF DETECTOR RESPONSE FOR DIFFERENT NEUTRON ENERGIES OF THE DETECTOR: A HIGHLY SENSITIVE SILVER-ACTIVATION DETECTOR

A.M. Shehada, I. Pyatkov

National Research Tomsk Polytechnic University,

Russia, Tomsk, Lenin Ave., 30, 634050

E-mail: shihada@tpu.ru

In this work, a simulations using Monte-Carlo code MCNP is used to simulate the fast neutron detector indicated in the paper [1] and also different materials and designs to get the most efficient design of detecting and registering signals due to fast neutrons interaction within the active materials (silver an others) and the plastic scintillator (here is NE-110). This detector consists of 31 silver foils (101.6 x 203.2 x 0.245 mm) sandwiched between 32 plates of NE-110 plastic scintillator, each 102.0 x 204.0 x 3.2 mm.

In the MCNP simulations, the following parameters were used in the input file: 1. The Neutron flux produced by 2 μ A deuteron ions (13.6 MeV) collided with beryllium target (compressed powder 3-mm thick 11-cm in diameter) equal to $(2.93 \times 10^{11} \text{ n/s})$. 2. The detector-cell «cell of collected energy and charge resulted from neutron interactions with detector materials » in the input file chosen to be filled with Air. 3. The plastic scintillator used in simulations is the Anthracene C₁₄H₁₀ (density = 1.28 g. cm⁻³ at 25 °C).

The results of the detector response (64 sheets of silver + scintillator) as a function of neutron energies for two cases: 1. For 10 cm paraffin moderator in front of the source and 5 cm for paraffin moderator on the sides of the detector, 2. For 15 cm paraffin moderator in front of the source and 5 cm for paraffin moderator on the sides of the detector.